How to predict the probability of a major nuclear accident after Fukushima Dai-ichi?

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CERNA Mines ParisTech

March 14, 2012

François Lévêque and Lina Escobar How to predict the probability of a major nuclear accident after Fukushima Da

Outline

Motivations Probability of nuclear accident: basic models Addressing the issues Conclusions

Outline



Probability of nuclear accident: basic models

- Frequentist approach
- Bayesian approach
- Issues

3 Addressing the issues

- Allowing safety progress
- Dealing with independence

4 Conclusions

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Motivations

- Who is wrong? Discrepancy between observed and estimated frequency of major nuclear accidents (Core damage frequency CDF, Large Early Release Frequency LERF)
 - CDF: Observed frequency is $\frac{11}{14.400}$ =7.6E-04/r.y versus the estimated 1.0E-04/r.y
 - LERF: Observed frequency is $\frac{4}{14.400}{=}2.8E{-}04/r.y$ versus the estimated 1.0E-06/r.y
- How to combine observations and other pieces of knowledge (e.g: probabilistic safety assessments, data of safety improvements) to predict the probability of a major accident?
- How does the probability of major accident change due to the Fukushima Dai-ichi event?

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Frequentist approach Bayesian approach Issues

Binomial distribution

The first step to compute the probability is to assume a model of accident occurrence.

Assumption 1

Let Y be the number of nuclear accidents. We are going to assume that they come from a binomial distribution.

$$f(y) \sim B(k, p)$$
$$P(Y = k) = C_n^k p^k (1 - p)^{n-k}$$

Where:

- n : Number of trials = Years* Number of reactors
- k: Number of "success" = accidents
- p: Rate in which "successes" arrive= frequency

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Frequentist approach Bayesian approach Issues

Results

Table: Binomial distribution results

	Number of reactors	1-P(k=0)
Worldwide	433	0.28
Europe	143	0.10
France	58	0.04

If we use the CDF goal (p=1.0E-0.4) we find that the probability of an accident for the next year should be 0.00043 worldwide. How can we explain such a difference?

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Uncertainty in the parameter

- In the previous computations, we have assumed that the rate in which the accidents arrive, given by p equals the observed frequency. So the probability is only determined by what we have observed so far ⇒
 Frequentist approach
- An alternative methodology consists in assuming that *p* is a random variable, to represent the fact that our parameter is uncertain.

Frequentist approach Bayesian approach Issues

Bayesian inference

The Bayesian approach relies upon Bayes' law to make consistent inferences about the plausibility of an hypothesis given new information. We have three main ingredients:

- The prior distribution: Encodes the information about the parameter's state of knowledge
- Observations: Allow to update the hypothesis that we have made in the prior
- Bayes' law

Let's recall Bayes' Law

$$p(H|e)rac{p(H)*p(e|H)}{p(e)}$$

Where:

- H: Hypothesis
- e: Evidence or data

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Bayesian updating

Under this approach, we take the Prior distribution for our parameter $\pi_0(p)$, and then we update it with the available data y using Bayes' law.





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Frequentist approach Bayesian approach Issues

Binomial-Beta Model

Some intuition about the Bayesian update with the binomial distribution. If the initial rate at which accidents arrive is given by:

$$p_0 = \frac{k}{n}$$

Where:

- k: Number of successes
- n: Number of trials

How does p_0 change if we add 2 more trials and we get 1 success?

$$p_1 = \frac{k+1}{n+2}$$

So if instead of 2, we get a proportion of t successes in s trials, we have:

$$p_1 = rac{k+st}{n+s}$$

Frequentist approach Bayesian approach Issues

Binomial-Beta Model

Taking into account the previous slide s and t represent our initial hypothesis about nuclear accidents.

- t will represent the expected rate
- s is the strength of our prior

This information will be encoded in the conjugate distribution of the binomial \Rightarrow Beta distribution

Assumption 2

We are going to assume that p follows a Beta distribution with parameters st, (1-t)s

$$\pi_0(p) = B\left[st, s(1-t)\right]$$

Where:

•
$$E(p) = t$$

• Var(p) = t(1-t)/(s+1)

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Binomial-Beta Model

Result

Given that the Beta distribution is the Binomial conjugate, the posterior distribution $\pi_1(p)$ is also a Beta distribution and the parameters are updated following a simple formula:

$$\pi_1(\boldsymbol{p}|\boldsymbol{y}_1) = \boldsymbol{B}\left[\alpha_1,\beta_1\right]$$

Where:

•
$$\alpha_1 = st + y_2$$

• $\beta_1 = s(1-t) + n - y_1$

Using the Beta properties we find that:

$$\mathrm{E}(p|y_1) = \frac{y_1 + st}{n+s}$$

Does it look familiar? Yes is the same result when adding st virtual successes in s trials.

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The prior distribution

How much do the results depend on the assumed prior?



Strong prior

Weak prior

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Bayesian approach

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PSA results to construct our Beta prior

Which values of t and s we have to use in $\pi_0(p)$?. We are going to use the PSA core damage frequency results. The estimated CDF is going to be t and using the 95% quantile we can recover the s.

- The NUREG 1150 (1990) has a average CDF equal to 8,9E10-5
- The NUREG 1560 (1997) has a average CDF equal to 6,5E10-5



PSA	t	S
— NUREG 1150	8,9E10-5	21882
— NUREG 1560	6,5E10-5	24869

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Bayesian results



Bayesian updating (Prior 1997)

	E(p)
Prior	6.50E10-5
Before Fukushima	2.56E10-4
Posterior Fukushima	3.21E10-4
Fukushima effect $\Delta\%$	0.2539

The results indicate that Fukushima Dai-ichi accident have updated the expected frequency of a nuclear accident by 25%

Frequentist approach Bayesian approach Issues

Assumptions underlying the probability distribution

If we compute the probability of at least one core damage in the following next year we find that:

	$\begin{array}{c c} 1 - P(k = 0) \\ \hline \text{Worldwide} & \text{Europe} & \text{France} \end{array}$		
Binomial	0.28	0.100	0.040
Bayesian	0.12	0.044	0.018

These results depend on strong assumptions underlying the models.

- Intermediate to the second term of te
- There has not been any safety progress during the development of nuclear power
- The distribution assumes that the events are independent
- All the reactors are the same: (i.e.,same probability regardless age, technology, localization)

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Allowing safety progress Dealing with independence

U.S PSA results



Figure: Core Damage Frequency Industry Average Trend. EPRI(2008)

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Individual PSA results



Figure: Core Damage Frequency. Mohrbach(2011)

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Safety improvements



Main feature:

 Most of the accidents were observed during the first years. When nuclear industry cumulated experience exponentially only Fukushima has been observed.

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Poisson regression

Assumption 1

Let y_t be the number of accident observed at time t. We are going to assume that they drawn from a Poisson distribution

$$f(y_t|\lambda) = rac{\exp(-\lambda E_t)(-\lambda E_t)^{y_t}}{y_t!}$$

Where:

- λ is the rate in which accidents arrive (accidents per reactor year)
- E_t is the exposure time at year t, corresponds to the number of operative reactors in each year

Assumption 2

We are going to assume that the arrival rate is defined as a log-linear link function of the unexpected unavailability factor (UUF)

$$\lambda = \frac{\exp(X_t'\beta)}{E_t}$$

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Poisson regression with UUF

- We choose the average Unplanned Unavailability Factor (UUF) as the explanatory variable
- It is the ratio between the amount of energy loss due to unplanned events in the plant with respect to the energy that the reference unit power could have produce during the same period



Table: Poisson with UUF

	Coefficients	Estimate Std.	Error z value	$\Pr(> z)$	
(Intercept)	-8.87689	0.68970	-12.8706	<2e-16	***
uuf	0.11455	0.06257	1.8308	0.06713	

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Event count time series model

- When we assume that we have an independent and identically distributed (i.i.d) sample, we give the same weight to each observation ⇒ In the Poisson regression all the accidents are equally important in the estimation
- But we have a time series, thus if events that we observe today are somehow correlated with those in the past ⇒ We should give more weight to recent events than those in the past.
- We propose to use a structural event-count time series model. This framework has been developed by Harvey and Fernandes (1989) and Brandt and Williams (1998)
- This model is called Poisson Exponentially Weighted Moving Average (PEWMA)

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PEWMA model

PEWMA model has a time changing mean λ_t that is described by two components:

Observed component: Given by a log-link as in Poisson regression that contains the explanatory variables that we observe at time t. We are interested in knowing how these variables affect the current state, which are represented with β coefficients

Unobserved component: Shows how shocks persist in the series, therefore it captures data dependence across time. This dependence is represented by a smoothing parameter defined as ω

 ω is the key parameter of PEWMA model, because it represents how we discount past observations in current state.

- $\bullet~$ If $\omega \to 0$ this means that the shocks persist in the series. We have high dependence in the data
- If $\omega
 ightarrow 1$ this will indicate that events are independent

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Bayesian approach in PEWMA model

To find the density of the parameter across time we use a Kalman filter that recursively uses Bayesian updating. The procedure consists in combining a prior distribution $\Gamma(a_{t-1}, b_{t-1})$ with the transition equation to find $\pi(\lambda_t|Y_{t-1})$ that is a Gamma distribution

$$\lambda_t | Y_{t-1} \sim \Gamma(a_{t|t-1}, b_{t|t-1})$$

Where:

•
$$a_{t|t-1} = \omega a_{t-1}$$

• $b_{t|t-1} = \omega b_{t-1} \frac{\exp(-X'_t \beta - r_t)}{F_t}$

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Bayesian approach in PEWMA model

Since the Gamma is the conjugate distribution for a Poisson likelihood, we use Bayes' law and the posterior distribution is also Gamma.



The Gamma parameters are updated following a simple formula:

 $\lambda_t | Y_t \sim \Gamma(a_t, b_t)$

Where:

- $a_t = a_{t|t-1} + y_t$
- $b_t = b_{t|t-1} + E_t$

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Poisson vs PEWMA



▶ PEWMA Results

Model	$\hat{\lambda_{2011}}$	$\Delta_{2011-2010}\%$
Poisson	0.00026	2.76%
PEWMA	0.00196	270%
Δ	-86.4%	

- Poisson estimate does not react substantially to Fukushima Dai-ichi, whereas in PEWMA model it is the most important event to predict the arrival rate
- The Poisson arrival rate is 86 % smaller than PEWMA rate for 2011
- Fukushima Dai-ich event represented an important increase in PEWMA estimation (270%)

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Probability of an accident and Fukushima Dai-ichi effect

Let's compare the results that from the models that we have discussed so far. When we compute at the end of 2011, the worldwide (433 nuclear reactors) probability of at least one nuclear accident for next year and the increase in the rate due to Fukushima Dai-ichi we find the following results:

Model	Frequency p	Worldwide $1 - P(k = 0)$	Fukushima Effect
Binomial	0.00076	0.28	0.37
Bayesian Binomial Beta	0.00032	0.12	0.25
Model	Arrival rate λ	Worldwide $1 - P(k = 0)$	Fukushima Effect
Poisson regression	0.00026	0.0003	0.027
PEWMA	0.00196	0.0020	2.70

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Conclusions

- The PEWMA model is a first attempt:
 - i To reconcile the observed frequency and the estimated CDF provided by nuclear operators and safety authorities
 - ii To combine observations and other data related to safety. It is important to use other sources of information on nuclear risk to predict the probability of a major nuclear accident
 - ii To deal with the time series and count nature of our data
- According to this model the Fukushima Dai-ichi effect on the increase in predictive probability is large
 - This is not a contradiction with qualitative assessments. For instance in many countries other than Japan, regulatory capture and seismic underestimation prevail.
- There still are some limitations
 - i The low number of observations. We should use a broader definition of accident to have more observations (i.e INES 2+)
 - ii Is important to incorporate the heterogeneity across nuclear fleet (i.e localization, technology, age)

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Corchan Nuclear Accidents

Year	Location	Unit	Reactor type
1959	California, USA	Sodium reactor experi-	Sodium-cooled power reactor
		ment	
1966	Michigan, USA	Enrico Fermi Unit 1	Liquid metal fast breeder reactor
1967	Dumfreshire, Scotland	Chapelcross Unit 2	Gas-cooled, graphite moderated
1969	Loir-et-Chaire, France	Saint-Laureant A-1	Gas-cooled, graphite moderated
1979	Pennsylvania, USA	Three Mile Island	Pressurized Water Reactor
			(PWR)
1980	Loir-et-Chaire, France	Saint-Laureant A-1	Gas-cooled, graphite moderated
1986	Pripyat, Ukraine	Chernobyl Unit 4	RBKM-1000
1989	Lubmin, Germany	Greifswald Unit 5	Pressurized Water Reactor
			(PWR)
2011	Fukushima, Japan	Fukusima Daiichi Unit	Boiling Water Reactor (BWR)
		1	
2011	Fukushima, Japan	Fukusima Daiichi Unit	Boiling Water Reactor (BWR)
		2	
2011	Fukushima, Japan	Fukusima Daiichi Unit	Boiling Water Reactor (BWR)
		3	
			Cochran (2011)

Figure: Major nuclear accidents 1955-2011

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Poisson Exponentially Weighted Moving Average (PEWMA)

1. Measurement equation: Is the stochastic component $f(y_t|\lambda_t)$, we keep our assumption that y_t is distributed Poisson with arrival rate λ_t

$$\lambda_t = \lambda_{t-1}^* \frac{\exp(X_t'\beta)}{E_t}$$

The rate has two components:

- An unobserved component λ_{t-1}^*
- A log-link like in Poisson regression

PEWMA Model equations

2. Transition equation: Shows how the mean changes over time.

$$\lambda_t = \lambda_{t-1} \exp(r_t) \eta_t$$

- r_t is the rate of growth
- η_t is a random shock that is distributed $B(a_{t-1}\omega, (1-a_{t-1})\omega)$
- ω is weighting parameter. When $\omega \to 1$ observations are independent, $\omega \to 0$ the series is persistent
- 3. Prior distribution: Describes the initial state.

$$\lambda_{t-1}^* \sim \Gamma(a_{t-1}, b_{t-1})$$

• We are going to use the conjugate for the Poisson that a Gamma distribution

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Kalman filter procedure

We are interested in finding:

$$f(y_t|Y_{t-1}) = \int_0^\infty \underbrace{f(y_t|\lambda_t)}_{Measurement} \underbrace{\pi(\lambda_t|Y_{t-1})}_{Unknown} d\lambda_t$$

So to find $\pi(\lambda_t|Y_{t-1})$ we use a Kalman filter. Following these steps:

- **Observe and Serve 1** Combine the prior distribution of λ_{t-1} with the transition equation to find the distribution of $\lambda_t | Y_{t-1}$
- **②** Using the properties of the gamma distribution we find the parameters $a_{t|t-1}, b_{t|t-1}$
- **()** We use the Bayes' updating formula to compute the distribution of $\lambda_t | Y_t$ whenever the information set is available (i.e $\forall t < T$)
- This updated distribution becomes the prior in the next period and we repeat the previous steps

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Posterior distribution for λ_t

When we combine the transition function with the prior is possible to show that:

$$\lambda_t | Y_{t-1} \sim \Gamma(a_{t|t-1}, b_{t|t-1})$$

Where:

• $a_{t|t-1} = \omega a_{t-1}$

•
$$b_{t|t-1} = \omega b_{t-1} \frac{\exp(-X'_t \beta - r_t)}{E_t}$$

The posterior distribution is also Gamma and the parameters are updated following a simple formula:

$$\lambda_t | Y_t \sim \Gamma(a_t, b_t)$$

Where:

•
$$a_t = a_{t|t-1} + y_t$$

• $b_t = b_{t|t-1} + E_t$

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Log-likelihood function

Now we can compute $f(y_t|Y_{t-1})$ that is given by a negative binomial density function

$$f(y_t|Y_{t-1}) = \int_0^\infty f(y_t|\lambda_t)\pi(\lambda_t|Y_{t-1})d\lambda_t$$

=
$$\frac{\Gamma(\omega_{a_{t-1}+y_t})}{y_t!\Gamma(\omega_{a_{t-1}})}\{\omega_{b_{t-1}}\frac{\exp(-X'_t\beta-r_t)}{E_t}\}^{\omega_{a_{t-1}}}$$
$$\times\{E_t+\omega_{b_{t-1}}\frac{\exp(-X'_t\beta-r_t)}{E_t}\}^{-(\omega_{a_{t-1}+y_t})}$$

So the predictive joint distribution is

$$f(y_0, ..., Y_T) = \prod_{t=0}^T f(y_t | Y_{t-1})$$

The log-likelihood function is based on this joint density

$$\mathcal{L} = \log(f(y_0, ..., Y_T))$$

Results

Table: PEWMA results

	Coefficient	Std.Error	Z-Score
ω	0.8477619	0.01338405	63.341200
constant	-7.7252291	0.50184279	-15.393723
UUF	0.1588987	0.04180016	3.801391

- *ŵ* can be consider as a smoothing or discounting parameter.
- *ŵ* different from 1 means that we reject the independence hypothesis. Small values indicate more dynamics so nearest observations are more important than those in the past.
- β̂ estimates have the same sign that in the Poisson model, which confirms a decreasing time trend in nuclear accidents and reductions in UUF are linked with smaller nuclear risk.